Selected Papers from the European Commission CAST (CArbon-14 Source Term) Project—A Summary of the Main Results from the Final Symposium

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CARBON-14 RELEASE AND SPECIATION FROM CARBON STEEL IN HIGHLY ALKALINE CONDITIONS

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ABSTRACT. The release and the speciation of carbon species from irradiated JRQ carbon steel samples, representative of the reactor pressure vessel of Belgian nuclear power plants, were studied in a saturated portlandite aqueous solution, relevant for the Belgian Supercontainer design, as perceived for the geological disposal of high-level nuclear waste. To achieve this, we performed simple immersion and potentiostatic corrosion tests. In addition, the corrosion rate (which determines the ¹⁴C release) was estimated by measuring the release of ⁶⁰Co. Gas chromatography showed that during the static corrosion test, the carbonaceous species methane, carbon dioxide, ethene, and ethane were produced. Under the hypothesis that all the carbon released from the JRQ steel was transformed into carbon-base gaseous compounds, this corresponds to a corrosion rate of approximately 100 nm/yr, which is in good agreement with literature data.

KEYWORDS: carbon-14, carbon steel, corrosion, speciation, nuclear waste disposal.

INTRODUCTION

The Belgian concept for the geological disposal of high-level nuclear waste, the so-called Supercontainer design, foresees disposal of reactor pressure vessel (RPV) steels in a cementitious environment. In carbon steel, which is the most important material of the RPV, ¹⁴C is mainly produced by the thermal neutron activation of ¹⁴N, according to the transmutation reaction ¹⁴N(n,p)¹⁴C. In the safety assessment of the geological disposal of high-level nuclear waste, ¹⁴C is a critical nuclide, because of its long half-life (5730 yr) and its high mobility in the geosphere and biosphere. The release of ¹⁴C is determined by the corrosion rate of carbon steel, which according to literature data is approximately 10–100 nm/yr (Grauer 1991; Fujiwara 2002; Mihara 2002; Smart 2009; Diomidis 2014; Kursten 2014).

In order to investigate the release of carbon-14 from carbon steel in a highly alkaline environment, we performed simple immersion tests and potentiostatic corrosion tests. The simple immersion tests consisted of exposing the sample to an anaerobic cementitious environment (saturated portlandite pore water solution) for several months without applying any potential, while in the shorter potentiostatic corrosion tests an electrochemical setup was used to fix the potential at a more anodic value than the corrosion potential, in order to accelerate the corrosion process. The carbon speciation was determined by chromatographic analyses of gases produced during the corrosion tests, while the corrosion rate was calculated by the ⁶⁰Co and the carbon release.

EXPERIMENTAL

Materials

The material used for our study was JRQ steel, which is an internationally accepted reference material representative for RPV steel. Its manufacturing process and general properties are

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Table 1 Nominal composition of JRQ steel (ASTM A533).

Element	C	Si	Mn	P	S	Cr	Mo	Ni
wt.%	0.10-0.17	0.60-0.90	0.50-0.80	0.025	0.035	0.50-0.75	0.15-0.25	<0.25

described in detail elsewhere (IAEA 2001). JRQ is the IAEA designation for the ASTM A533 grade B class 1 steel. Its composition is given in Table 1. The irradiated material was representative for end-of-life conditions in the Belgian nuclear power plants and as such offered the advantage of providing realistic data on carbon-14 release and speciation. The investigated samples were irradiated in 1993 in the BR2 reactor at SCK•CEN in the Vestale irradiation loop at a temperature of 290 \pm 15°C and a pressure of 150 bar. For the accelerated tests, a sample with dimensions $10\times6\times4.5$ mm was used with a contact dose rate of 920 μ Sv/h. For the static tests, three samples with dimensions of approximately $8\times10\times1$ mm and a contact dose rate of approximately 350 μ Sv/h were used.

Nitrogen Content Analysis and γ-Ray Spectrometry

The nitrogen content of unirradiated JRQ steel was measured by using an inert gas fusion method with a LECO TC436 analyzer. The sample was cut into a small cube of \sim 0.5 g and stored in acetone until the analysis. Before the sample analysis, 3 nitrogen blanks were measured and the results were normalized with the blanks. The instrument was calibrated using two certified reference materials of different nitrogen concentrations: AR668 with a nitrogen concentration of 29 μ g/g (Alpha Resources) and AR660 with a nitrogen concentration of 61 μ g/g (Alpha Resources). The lower limit of detection of the method was approximately 1 μ g/g.

For γ -ray spectrometry, two HPGe detectors were used, one from Canberra and one from Ortec. The HPGe detectors were energy- and efficiency-calibrated over an energy range of 60–2000 keV using a γ -ray reference solution with a mixture of 10 different radionuclides.

Corrosion Experiments

Two types of corrosion experiments have been performed: simple immersion tests and potentiostatic corrosion tests. The simple immersion tests were long-term leaching experiments that allowed for the observation of a realistic corrosion behavior and associated carbon-14 release and speciation. The potentiostatic corrosion tests, which involved applying a fixed potential to a JRQ steel electrode, served mainly for preliminary speciation determination. Indeed, by enforcing a higher corrosion rate, the yield of released carbon species was increased.

Choice of the Electrolyte

For the static and accelerated test, the main electrolyte used was a saturated portlandite $(Ca(OH)_2)$ solution of pH 12.5. Portlandite is representative for the Belgian waste repository environment, which involves the use of large quantities of Ordinary Portland Cement (OPC). The artificial pore water was prepared in a glove box under anaerobic conditions and degassed with nitrogen to reduce the risk of dissolving environmental $^{14}CO_2$, which would cause an artifact. In the accelerated tests, a second type of electrolyte was used, by adding 0.5 M CaCl₂ (this corresponds to 1 M of chloride) to a saturated portlandite solution. This increased the production of carbon species and made their detection easier.

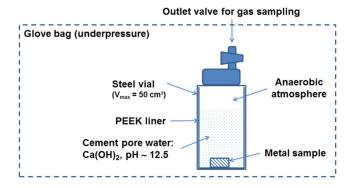


Figure 1 Schematic of the experimental setup for the simple immersion tests.

Corrosion Setups and Sampling

Figure 1 shows the schematic of the experimental setup for the simple immersion tests. The JRQ steel sample was placed in contact with portlandite pore water inside a steel vial equipped with a PEEK liner. The internal volume of the steel vial was 50 mL. The vials were each filled with 35 mL of saturated portlandite pore water before adding the JRQ steel sample. Afterwards, the lid was screwed to the vial to make it airtight. All these manipulations took place in a glove bag under nitrogen atmosphere to ensure that conditions for the static tests were anaerobic. After the leaching tests, which lasted for approximately 7.5 months (231 days), gas and liquid samples were taken and analyzed for carbon species. The gas sampling was performed with a 500 μL gas tight syringe, which was then transferred to the gas chromatograph for analysis.

For the accelerated corrosion tests, JRQ steel specimens were embedded in an epoxy resin and mechanically wet-ground, with successively finer SiC papers, down to 500 grit, and then cleaned with double demineralized water and finally left to dry in an argon atmosphere. The electrochemical test cell consisted of a glass vial equipped with three electrodes attached to the lid: (1) the working electrode (carbon steel sample), (2) the platinum counter electrode, and (3) the inhouse manufactured Ag/AgCl reference electrode. These electrodes constitute a traditional three-electrode setup described in the general literature on electrochemistry (Tait 1994). Prior to the actual accelerated tests, a polarization curve was recorded. From this polarization curve, a potential was derived (resulting in an increased current density or corrosion rate) which was applied to the JRQ steel electrode during the accelerated test.

Determination of the Carbon Speciation

The carbon speciation was determined using a gas chromatography system type Shimadzu GC-2010 Plus, tailor-made to our needs. The carrier gas was pure Argon (N7.5, Air Products).

Detection limits for common carbon compounds, measured with the Shimadzu Table 2 GC-2010 Plus.

Gases	Reporting limits (µg/g)
Methane	0.4
Carbon dioxide	3
Ethene	0.2
Ethane	0.2
Propene	1.5
Propane	0.75

To remove any residual impurities, this gas was passed through two gas purifiers (VICI Valco Instruments Co. Inc.) before it entered the GC column. For the analyses we used a ShinCarbon ST column composed of a high surface area carbon molecular sieve. This has been developed for the rapid separation of permanent gases, such as N₂, O₂, CO, and CO₂, and low hydrocarbons, without cryogenic cooling. The detector was a Pulsed Discharge Helium Ionization Detector (PDHID). Table 2 shows the detection limits for common carbon compounds.

RESULTS AND DISCUSSION

Estimation of the ¹⁴C Content of Irradiated JRQ Steel

The basis for the estimation of the ^{14}C content of irradiated JRQ steel was the determination of the ^{14}N content of an unirradiated sample, assuming that the neutron activation of ^{14}N is the most probable production pathway of ^{14}C in carbon steel. We obtained an average ^{14}N concentration of $19\pm0.4~\mu g/g$ in the unirradiated material, which is slightly lower than the range mentioned in literature (20–30 $\mu g/g$; IAEA 2009). Assuming the measured value, and taking into account the irradiation history with a total neutron fluence of $5\times10^{19}~n/cm^2$, the theoretical concentration of ^{14}C in irradiated samples was calculated using the "Triton" depletion sequence in the SCALE-6.1 code. This yielded a ^{14}C concentration of 1.069×10^3 wt. $\mu g/g$, which corresponds to an activity of 175 Bq/g. The corrosion of 1 g of JRQ steel will lead to the release of approximately 1 ng of ^{14}C .

Corrosion Rate Determination from ⁶⁰Co Measurements

The measurement of the corrosion rate of carbon steel in a highly alkaline environment is a difficult task with traditional techniques because of the extremely low values involved. Therefore, we investigated whether the corrosion rate could be determined radiometrically by measuring the release of activation products, such as ⁶⁰Co, when an irradiated steel sample is in contact with artificial portlandite pore water. This approach assumes a homogeneous distribution of ⁶⁰Co in the material and the same leaching rate as Fe under alkaline conditions.

The corrosion rate of carbon steel in highly alkaline solutions is determined by the diffusion of iron through a passive film on top of the steel surface (Kursten 2014). This implies that the corrosion rate obeys a parabolic law, and this behavior was the basic assumption for our interpretation of the results from the 60 Co determinations. Table 3 summarizes the results from γ -ray spectrometry, showing the total 60 Co activity in a 20 mL test cell. The result at 50 days, being two orders of magnitude higher than the other values, is presumed to be an outlier. This might have been caused by the accidental transfer of a very small piece of irradiated metal into the measurement vial. Therefore, we omitted this data point from subsequent analyses.

Table 3 Total 60 Co activity released in the 20 mL test cell (results obtained by γ -ray spectrometry).

Elapsed time (days)	Activity (Bq)	2 σ (Bq)	
2	0.236	0.040	
9	0.556	0.058	
16	0.316	0.044	
23	0.826	0.074	
30	0.580	0.056	
50	230	16	

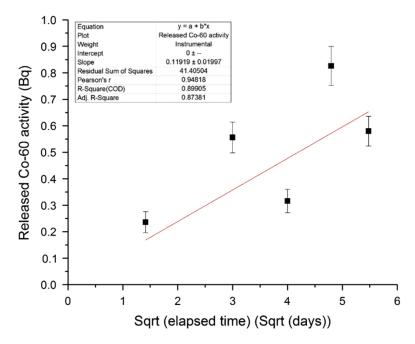


Figure 2 Linear fit of the released ⁶⁰Co activity as a function of the square root of the elapsed time.

The scatter on the results is high. This might be due to the precipitation of small sized particles, which are visible in the solution. Indeed, under alkaline conditions, the released 60Co can precipitate as a hydroxide. It was difficult to completely homogenize the solution, which made it difficult to take representative samples. Another source of error might be that these precipitates adsorbed on the wall of the test cell. The third source of error may lie in the γ-ray spectrometry itself. In order to obtain a reliable measurement, a prerequisite is that ⁶⁰Co is homogenously distributed throughout the solution, while in our case the precipitates may have (partially) settled on the bottom of the vial. In this case, the calibration factor can differ by a factor of two. Furthermore, the amount of precipitates in the vial may have been different for each sampling, and it is not possible to quantify this retroactively.

Figure 2 shows a linear fit of the ⁶⁰Co release data as a function of the square root of the elapsed time of the experiment, with the fit line forced through the zero point. A fit with $R^2 = 0.91$ is obtained. The parabolic rate law is

$$R_{Co-60} = (0.12 \pm 0.02)\sqrt{t} \tag{1}$$

where R_{Co-60} is the total released 60 Co activity in Bq at time t, and t is the elapsed time in days. This confirms that the release of 60 Co is controlled by diffusion through the passive layer formed under highly alkaline conditions.

After 365 days, according to Equation (1), 2.3 Bq of ⁶⁰Co will have been released. The original carbon steel specimen had dimensions of $2 \times 2 \times 35$ mm and a total activity of 1.6 x 10^5 Bq. This means that the dissolved fraction of the specimen (assuming that ⁶⁰Co was distributed evenly throughout the carbon steel sample) was $2.3 / 160,000 = 1.4 \times 10^{-5}$. On a total initial volume of $140 \,\mathrm{mm}^3$ this results in $2 \times 10^{-3} \,\mathrm{mm}^3$ of dissolved carbon steel. The exposed surface of the sample is 288 mm². This yields a corrosion depth, after one year of exposure, of 7 nm. A corrosion rate of 7 nm/yr is a bit lower than the reference range of 10–100 nm/yr, but this may be explained by the observed precipitation. Also, it is not proven that the diffusion behavior of cobalt through the passive film is the same as that of iron. It probably also means that in these conditions ⁶⁰Co measurements are not the ideal way to determine the corrosion rate of carbon steel.

Speciation Determination

Simple Immersion Tests

After immersing an irradiated JRQ steel sample in saturated portlandite pore water, we applied a potential of 620 mV (vs. SHE) to the sample. This resulted in a current density of 5.9×10^{-6} A. The test lasted for 165 h and gas samples were taken after respectively 25, 64, and 165 hr. GC analysis showed the production of hydrogen gas by anaerobic corrosion of the steel and the appearance of CO_2 . No other gaseous carbon species were detected.

For the second test we added 0.5 M CaCl_2 to the saturated portlandite pore water to induce pitting corrosion and thus to increase the yield of corrosion products. A potential of -200 mV (SHE) was applied to the sample because this corresponds to the highest current density within the stability domain of water. The test lasted for 4.5 hr. Again, the production of H_2 and CO_2 was detected but no other carbon species were measured.

Potentiostatic Corrosion Tests

Three irradiated samples were used for the static corrosion tests, with respective codes CS-A, CS-B, and CS-C. After 231 days, gas samples were taken. Figure 3 shows the corresponding gas chromatograms. At a retention time of approximately 1 minute, a high nitrogen/argon/oxygen peak appears. Such a high concentration of O_2 could not have come from an accidental injection of air in the GC column, because its presence is recurrent in all three samples. Moreover, a leak in the vials can be excluded because the CO_2 signal is comparable to the one obtained during calibration of the GC. The third and most probable explanation is that the oxygen peak originates from the radiolysis of water by the irradiated JRQ sample.

Further peaks can clearly be distinguished for hydrogen, methane, carbon dioxide, ethene, and ethane. For two of the three chromatograms, the ethane peak is largely hidden by a large peak, which could be attributed to water. Calibration curves were used to quantify the signals for the carbon compounds. The results of this quantification are shown in Table 4. The measured concentrations are low, but well above the detection limit. On the basis of the calculated carbon-based gas production and the carbon concentration in the bulk of the JRQ steel, an

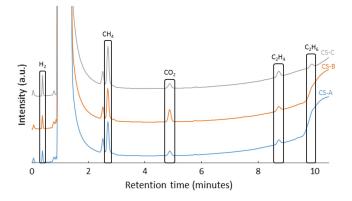


Figure 3 Gas chromatograms obtained after the simple immersion tests with irradiated JRQ carbon steel in portlandite pore water (duration 231 days).

Methane Ethene Ethane Calculated Calculated Calculated concentration concentration concentration Peak Peak Peak ID area μL/L mol/L area μL/L mol/L area μL/L mol/L $9.9 \pm 0.5 \ 6.1 \times 10^{-9} \ 144299 \ 1.6 \pm 0.1 \ 1.0 \times 10^{-9}$ CS-A 681880 $9.6 \pm 0.5 \ 5.9 \times 10^{-9} \ 137950 \ 1.5 \pm 0.1 \ 0.9 \times 10^{-9}$ CS-B 660379 $817457 \ 11.9 \pm 0.6 \ 7.3 \times 10^{-9} \ 189461 \ 2.1 \pm 0.1 \ 1.3 \times 10^{-9} \ 94967 \ 0.6 \pm 0.1 \ 0.4 \times 10^{-9}$ CS-C 6.4×10^{-9} 1.1×10^{-9} 10.4 1.7 0.4×10^{-9} Mean 0.6 value

Calculated concentrations of methane, ethene, and ethane in the gas phase after simple immersion tests with irradiated JRQ carbon steel in portlandite pore water (duration 231 days).

approximation of the corrosion rate could be determined. Under the hypothesis that all the carbon released from the JRQ steel was transformed in carbon-based gaseous compounds, the calculated corrosion rate was 68–117 nm/yr, which is in good agreement with the 10–100 nm/yr reference range.

CONCLUSIONS

The carbon-14 release and speciation of irradiated carbon steel representative of the reactor pressure vessel of Belgian nuclear power plants were investigated in a cementitious environment relevant for the Supercontainer design, as perceived for the geological disposal of high-level radioactive waste in Belgium. To achieve this, two types of corrosion tests were performed: (1) simple immersion tests to obtain information on the carbon steel behavior in real disposal conditions and (2) potentiostatic corrosion tests to obtain in a shorter reaction time an indication of the corrosion mechanism and the speciation of light carbon molecules.

In order to determine the corrosion rate of carbon steel, we measured the ⁶⁰Co release by γ-spectrometry. In spite of the large scatter on the results, we could confirm a parabolic law rate for the release of 60Co, which is in accordance with literature results. This led to a corrosion rate of 7 nm/yr, which is relatively low in comparison with earlier data. The simple immersion tests showed that hydrogen, methane, carbon dioxide, ethene, and ethane were released from the carbon steel. Under the hypothesis that all the carbon released from the carbon steel was transformed into carbon-base gaseous compounds, the calculated corrosion rate was 68-117 nm/yr, which is in good agreement with literature data.

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REFERENCES

- Diomidis N. 2014. Scientific basis for the production of gas due to corrosion in a deep geological repository. *NAGRA*, *Switzerland*, report NAB 14-21.
- Fujiwara A. 2002. Gas generation by steel corrosion under reductive conditions – Continuous measurement test, presented at the GASNET Workshop, 12–14 November 2002, Cologne, Germany.
- Grauer R, Knecht B, Kreis P, Simpson JP. 1991. The long term corrosion rate of passive iron in anaerobic alkaline solutions, Werkstoffe und Korrosion 42:637–42.
- IAEA. 2001. Reference manual on the IAEA JRQ correlation monitor steel for irradiation damage studies, *IAEA*, *Austria*, IAEA-TECDOC-1230.
- IAEA. 2009. Integrity of reactor pressure vessels in nuclear power plants: assessment of irradiation embrittlement effects in reactor pressure vessel steels, *IAEA*, *Austria*, IAEA Nuclear Energy Series No. NP-T-3.11.
- Kursten B. 2014. Uniform corrosion rate data of carbon steel in cementitious environments

- relevant to the Supercontainer design, SCK•CEN, Belgium, State-of-the-art report as of December 2013, SCK•CEN report no. SCK•CEN-ER-94.
- Mihara M, Nishimura T, Wada R, Honda A. 2002. Estimation on gas generation and corrosion rates of carbon steel, stainless steel and zircaloy in alkaline solutions under low oxygen condition. *JNC Technical Review* 15:91–101.
- Smart NR, Rance AP, Winsley RJ, Fennell PAH, Reddy B, Kursten B. 2009. The effect of irradiation on the corrosion of carbon steel in alkaline media. In: L'Hostis V, Gens R, Gallé C, editors. Long-Term Performance of Cementitious Barriers and Reinforced Concrete in Nuclear Power Plants and Waste Management. NUCPERF 2009, RILEM proceedings PRO 64. p 45–52.
- Tait WS. 1994. An introduction to electrochemical corrosion testing for practicing engineers and scientists. USA, Pair O Docs Publications.