

J. Plasma Phys. (2023), vol. 89, 905890507 © The Author(s), 2023. Published by Cambridge University Press

This is an Open Access article, distributed under the terms of the Creative Commons Attribution licence (http://creativecommons.org/licenses/by/4.0/), which permits unrestricted re-use, distribution and reproduction, provided the original article is properly cited. doi:10.1017/S0022377823000934

Nonlinear saturation of resistive tearing modes in a cylindrical tokamak with and without solving the dynamics

J. Loizu¹⁰¹,† and D. Bonfiglio^{102,3}

¹École Polytechnique Fédérale de Lausanne, Swiss Plasma Center, CH-1015 Lausanne, Switzerland
 ²Consorzio RFX, CNR, ENEA, INFN, Università di Padova, Acciaierie Venete SpA, Corso Stati Uniti 4, 35127, Padova, Italy

³CNR-ISTP Padova, Corso Stati Uniti 4, 35127, Padova, Italy

(Received 31 May 2023; revised 21 August 2023; accepted 21 August 2023)

We show that the saturation of resistive tearing modes in a cylindrical tokamak, as well as the corresponding island width, can be directly calculated with a magnetohydrodynamics (MHD) equilibrium code without solving the dynamics and without considering resistivity. The results are compared with initial-value resistive MHD simulations and to an analytical nonlinear theory. For small enough islands, the agreement is remarkable. For sufficiently large islands, the equilibrium calculations, which assume a flat current profile inside the island, overestimate the saturation amplitude. On the other hand, excellent agreement between nonlinear resistive MHD simulations and nonlinear theory is observed for all the considered tearing unstable equilibria.

Key words: fusion plasma, plasma instabilities, plasma simulation

1. Introduction

In magnetic fusion, tearing modes are an important class of instabilities that can often be described by resistive magnetohydrodynamics (MHD) (White 2014). Tearing modes are current-driven internal instabilities that drive spontaneous magnetic reconnection at rational surfaces in toroidally confined plasmas. Magnetic islands grow in time, thus potentially affecting confinement, and eventually saturate nonlinearly. If their saturated amplitude is too large, they can trigger disruptions, sometimes even inducing the complete termination of the plasma discharge (La Haye 2006). While resistivity is required to allow these modes to grow, their saturated states are three-dimensional MHD equilibria that are often independent of resistivity, at least for sufficiently large Lundquist numbers (Poyé *et al.* 2014). This suggests that a direct MHD equilibrium computation might be able to describe and even predict these saturated states, as was shown to be the case for the saturation of ideal instabilities in tokamaks (Cooper *et al.* 2010, 2011; Kleiner *et al.* 2019) or the formation of saturated double-axis states in reversed field pinches (Dennis

† Email address for correspondence: joaquim.loizu@epfl.ch

et al. 2013b). The possibility of finding these nonlinearly saturated tearing modes directly, namely, without solving the dynamics and without free parameters, was investigated and successfully shown for the first time in a slab geometry (Loizu et al. 2020). More recently, a similar type of calculation was carried out in a cylindrical geometry for the case of externally forced reconnection in a cylinder (Wright et al. 2022), although the volume available for reconnection remained unconstrained. Here, we investigate the possibility of directly predicting the saturated state of tearing modes in a cylindrical tokamak geometry at zero pressure. In this configuration, we can exploit an exact nonlinear saturation theory (Arcis, Escande & Ottaviani 2006) that predicts the saturated island width for a given initially axisymmetric equilibrium that is unstable to a tearing mode. The results are compared with those obtained with the SpeCyl code (Cappello & Biskamp 1996; Bonfiglio, Chacón & Cappello 2010), which solves the time-dependent nonlinear visco-resistive MHD equations in a cylinder. These predictions serve as a benchmark for the direct equilibrium approach, where we use the stepped-pressure equilibrium code (SPEC) (Hudson et al. 2012, 2020) to describe, without free parameters, the spontaneous formation of these nonlinearly saturated tearing islands. In general, the direct calculation of saturated states with an equilibrium approach is interesting for two reasons: on the one hand, it provides insights into the nature of the saturated state itself; on the other hand, the approach is potentially much faster than the initial-value approach, especially as we consider larger systems (such as magnetic fusion reactors) or more complex geometries (such as stellarators), where analytical theories are not available and initial-value approaches are very demanding.

The remainder of this paper is organized as follows. In § 2 we construct and analyze the linear stability of a family of MHD equilibria both analytically and with SPEC. Section 3 describes a nonlinear theory that predicts the saturated island width of these unstable equilibria. In § 4 we carry out nonlinear simulations with the SpeCyl code and compare the obtained saturated tearing modes with the theoretical predictions. Section 5 describes how SPEC can also be used to retrieve the saturated states corresponding to each unstable equilibrium, and a discussion on the level of agreement is provided. Conclusions follow in § 6.

2. Family of equilibria and linear stability analysis

We construct a family of non-rotating, cylindrical MHD equilibria with zero pressure and parametrized by the on-axis safety factor q_0

$$q(r) = q_0 \left[1 + \left(\frac{r}{r_0} \right)^2 \right], \tag{2.1}$$

for $0 \le r \le a$ and where a and r_0 are fixed and correspond, respectively, to the radius of the plasma boundary and to the current channel width. As we will see, the value of q_0 controls the stability of the equilibrium, the presence and location of resonant rational surfaces as well as the width of the saturated islands. In the strong guide field limit $(B_z \gg B_\theta)$ where B is the magnetic field), which for a tokamak with $q \sim 1$ also corresponds to the large aspect ratio limit $(R \gg a)$ where R is the tokamak major radius), the axial current density profile associated with this q-profile is given by

$$j_z(r) = \frac{j_0}{\left[1 + \left(\frac{r}{r_0}\right)^2\right]^2},\tag{2.2}$$

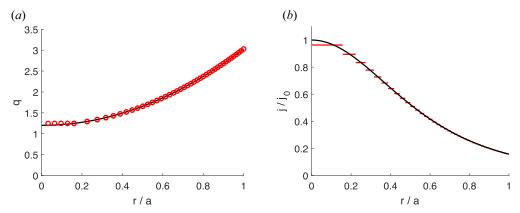


FIGURE 1. Equilibrium profiles for the safety factor (a) and current density (b) as given by (2.1)–(2.2) (solid black lines) and as obtained from SPEC (red circles and lines) with $N_v = 41$. Here, $q_0 = 1.2$ and $r_0/a = 0.81$.

where $j_0 = 2B_0/(\mu_0 q_0 R)$ is the on-axis current density and μ_0 the vacuum magnetic permeability, $B_0 = B_z(0)$ and $2\pi R$ is the length of the cylinder, which is considered to be 2π -periodic in both the polar angle θ and $\varphi = z/R$. Figure 1 shows examples of profiles corresponding to (2.1)–(2.2). Given the fixed, circular plasma boundary at r = a, and given q_0 , r_0 , B_0 and R, the ideal MHD equilibrium magnetic field B is fully determined by the force-balance equation $\mathbf{j} \times \mathbf{B} = \nabla p = 0$, which reduces to a one-dimensional, first-order differential equation for $B_z(r)$. These equilibria are ideally stable as long as $q_0 > 1$ (Freidberg 2014), but as we will see in the next section, they can be resistive unstable (Furth, Rutherford & Selberg 1973).

We can also construct these equilibria numerically by using SPEC (Hudson et al. 2012, 2020), with the purpose of seeking nearby, relaxed (lower potential energy) equilibria, which can be accessed via the possible opening of magnetic islands. SPEC considers a finite number N_v of nested volumes, hereafter referred to as relaxation volumes, that are separated by magnetic surfaces, hereafter referred to as ideal interfaces. In a cylindrical geometry, the position of each ideal interface l is generally described by $\mathbf{x}_l(\theta,\varphi) = r_l(\theta,\varphi)\cos\theta\hat{i} + r_l(\theta,\varphi)\sin\theta\hat{j} + R\varphi\hat{k}$, with $(\hat{i},\hat{j},\hat{k})$ the cartesian unit vectors, and thus can be determined by a set of Fourier coefficients, since we can write $r_l(\theta, \varphi) =$ $\sum_{m,n} r_{l,mn} \cos{(m\theta - n\varphi)}$. In each relaxation volume, the magnetic field solves a Beltrami equation $\nabla \times \mathbf{B} = \mu \mathbf{B}$ with μ a constant that is related to the parallel current density. The solution in the volumes is then determined by the shape of the boundary interfaces where $\mathbf{B} \cdot \mathbf{n} = 0$, the value of μ and two scalars Ψ_p and Ψ_t that correspond to the 'poloidal' (constant- θ) and 'toroidal' (constant- φ) magnetic fluxes enclosed by each volume (in the innermost volume, only Ψ_t is required). The position and shape of each interface is unknown a priori and is determined by a jump condition across each interface, $[[B^2]]$ $B_{+}^{2} - B_{-}^{2} = 0$, where + and - refers to each side of the interface. This condition is the local equivalent of force balance and ensures that the inner and outer pressures are in balance everywhere on an interface. We remark that equilibria with finite plasma pressure can also be described in SPEC but here we are considering zero-pressure equilibria. The magnetic fields satisfying these equations are extrema of the plasma potential energy, $W = \int_{V_0} B^2/(2\mu_0) \, dV$, when considering arbitrary variations in the magnetic field inside the relaxation volumes and ideal variations of the interface geometries, with the constraint of a conserved magnetic helicity, $K_l = \int_{V_l} \mathbf{A} \cdot \mathbf{B} \, dV$, and magnetic fluxes in each relaxation

volume (Hudson *et al.* 2012). Here, $V_p = \sum_{l=1}^{N_v} V_l$ is the total plasma volume and A is the magnetic vector potential. Furthermore, it has been shown that the magnetic field satisfying these equations retrieves exactly ideal MHD in the limit $N_v \to \infty$ (Dennis *et al.* 2013*a*). Hence, SPEC can be used to construct the family of ideal MHD equilibria described by (2.1) if a large value of N_v is chosen and if the appropriate constraints are provided. In fact, the constants (μ, Ψ_p, Ψ_t) in each volume, which determine the Beltrami fields, can be related to other physical quantities such as the safety factor q on each side of each interface, q_+ and q_- , or the net toroidal current flowing in each volume, I_{vol} , and on each interface, I_{surf} (Baillod *et al.* 2021). One can thus for example constrain in each volume (q_+, q_-, Ψ_t) if the safety factor is to be imposed, or $(I_{\text{vol}}, I_{\text{surf}}, \Psi_t)$ if the current profile is to be imposed. Figure 1 shows an example of an equilibrium reconstructed with SPEC, where $N_v = 41$ and the safety factor was constrained on each interface by following (2.1). After the SPEC calculation, one can look at the global behaviour of q(r) as well as that of $j_{\bar{c}}(r)$, which indeed reproduce well the analytical profiles.

2.1. Linear stability from classical tearing theory

The linear resistive stability of a cylindrical tokamak with zero pressure is uniquely determined by the sign of Δ' (Furth *et al.* 1973)

$$\Delta' = \lim_{\epsilon \to 0^+} \frac{\hat{\psi}'(r_s + \epsilon) - \hat{\psi}'(r_s - \epsilon)}{\hat{\psi}(r_s)},\tag{2.3}$$

with $\Delta' > 0$ implying instability. Here, $\hat{\psi}(r)$ is the amplitude of the perturbed flux function $\psi(r,\theta,\varphi) = \hat{\psi}(r)\cos(m\theta - n\varphi)$ for the linear ideal mode (m,n) associated with the rational surface $q(r_s) = q_s = m/n$, where m and n are, respectively, the poloidal (along θ) and toroidal (along φ) mode numbers. The corresponding magnetic field perturbation is $\delta \mathbf{B} = \nabla \psi \times \hat{\mathbf{z}}$, and thus the radial magnetic field perturbation, δB_r , for this mode is proportional to $\hat{\psi}$. Notice that Δ' depends on equilibrium profiles but is independent of resistivity. It can be calculated from the linearized ideal MHD equations, by matching the so-called 'outer solutions'. A shooting code provides a numerical solution for $\hat{\psi}(r)$ and a value of Δ' for a given resonant surface. For the family of equilibria described by (2.1), choosing $r_0/a = 0.81$, R/a = 10 and assuming the presence of a perfectly conducting shell at the plasma boundary, we find instability for the m=2, n=1mode whenever $1 < q_0 < 2$. In that range of q_0 values, we also find no other instability, hence, we can focus on the $q_s = 2$ resonant surface. Figure 2 shows an example of solution for $\hat{\psi}(r)$ for $q_0 = 1.2$. Figure 3 shows the obtained values of Δ' as a function of q_0 . For larger values of q_0 , the value of Δ' keeps increasing but of course for $q_0 > 2$ there is no instability because there is no $q_s = 2$ surface anymore.

2.2. Linear stability from SPEC

The stability of each of these equilibria can also be assessed with the SPEC code by evaluating the eigenvalues of the Hessian matrix \mathcal{H} , whose coefficients measure the second variation of the plasma potential energy with respect to perturbations in the geometry of the ideal interfaces, with the constraint of fixed magnetic helicity and fluxes in each volume (Loizu & Hudson 2019; Kumar *et al.* 2021, 2022). In a cylindrical geometry, however, one can also directly evaluate the eigenvalues of the force-gradient matrix \mathcal{G} , whose coefficients measure the variation in the force on each interface, $f_l = [[B^2/2\mu_0]]_l$, with respect to perturbations in the geometry of the ideal interfaces, $r_{l,mn}$, with the constraint of fixed magnetic helicity and fluxes in each volume (see Appendix A for more

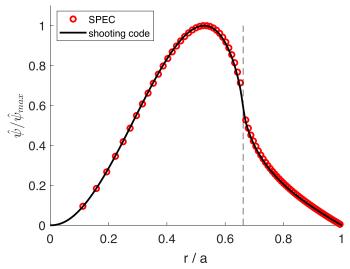


FIGURE 2. Radial profile of the perturbed flux function $\hat{\psi}$ for the unstable m=2, n=1 mode obtained from the linearized ideal MHD equations (black solid line). The same function can be inferred from the linear radial displacement $\xi(r)$ obtained with SPEC using the relation $\hat{\psi}(r) = rB_{z0}(1/q(r) - 1/q_s)\xi(r)/R$. The initial equilibrium is the same as for figure 1. The dashed line indicates $r = r_s$.

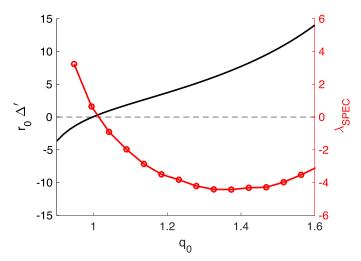


FIGURE 3. Value of Δ' vs q_0 as obtained from a shooting code (black solid line) for the m=2, n=1 mode. The equilibrium q profile is given in (2.1) with $r_0/a=0.81$ and R/a=10. The smallest eigenvalue of the force-gradient matrix from SPEC, $\lambda_{\rm SPEC}$, is also shown in arbitrary units for each value of q_0 (red circles).

details). Let us call λ the smallest eigenvalue of \mathcal{G} . If $\lambda \geq 0$, the equilibrium is stable. If $\lambda < 0$, then the equilibrium is unstable and the eigenmode $\mathbf{u}_{\lambda} \equiv \{r_{l,mn}\}$ corresponding to the eigenvalue λ provides information about the mode number (m, n) of the instability as well as its radial structure. For the family of equilibria described by (2.1), we find that, for $1 < q_0 < 2$, there is a negative eigenvalue (thus instability) that corresponds to the m = 2, n = 1 mode (figure 3). Outside this interval of q_0 -values, all eigenvalues are positive

(thus stability). In the case of instability, the radial structure of the unstable eigenmode agrees well with the one obtained from the shooting code, as shown in figure 2. Indeed, this comparison is carried out by relating the radial displacement of the flux surfaces to the perturbed flux function, $\delta B = \nabla \times (\xi \times B_0) = \nabla \psi \times \hat{z}$, and evaluating the radial displacement of the SPEC ideal interfaces, which has an amplitude $\xi(r_l) = r_{l,mn} - r_{l,00}$ for each mode (m, n).

3. Nonlinear saturation theory

The growth of the tearing mode promotes the growth of a magnetic island. The rate γ at which the island grows depends on the plasma resistivity η , both in the linear (Furth *et al.* 1973) and nonlinear (Rutherford 1973) phases. The saturation of the tearing mode, which in itself proves the existence of a nearby, generally three-dimensional MHD equilibrium, can be characterized by the width of the saturated island, w_{sat} , and that value becomes independent of resistivity for sufficiently small resistivity (Poyé *et al.* 2014). For a cylindrical tokamak, Arcis *et al.* (2006) derived a nonlinear theory for the time evolution of the island width w(t)

$$\frac{\mu_0}{\eta_{\text{eq}}(r_s)} \frac{\mathrm{d}w}{\mathrm{d}t} = 1.22\Delta' + w \left\{ \frac{\mathcal{A}^2}{2} \ln \frac{w}{w_0} - 2.21\mathcal{A}^2 + 0.40 \frac{\mathcal{A}}{r_s} + \frac{\mathcal{B}}{2} + 0.17\sigma \frac{\mathcal{A}^2 s}{2 - s} \right\}, \quad (3.1)$$

that is exact to first order in w. The coefficients in (3.1) can all be calculated from the equilibrium profiles, namely,

$$\mathcal{A} = \frac{j'_{\text{eq}}(r_s)}{j_{\text{eq}}(r_s)} \left(1 - \frac{2}{s} \right); \quad \mathcal{B} = \frac{j''_{\text{eq}}(r_s)}{j_{\text{eq}}(r_s)} \left(1 - \frac{2}{s} \right); \quad w_0 = a \exp\left(-\frac{\Sigma'}{2\mathcal{A}} \right), \quad (3.2a-c)$$

where $j_{eq}(r)$ is the equilibrium current density profile, which in our case is given by (2.2), $s = r_s q'(r_s)/q(r_s)$ is the magnetic shear at the resonant surface,

$$\Sigma' = \lim_{\epsilon \to 0^{+}} \left[\frac{\hat{\psi}'(r_{s} + \epsilon) + \hat{\psi}'(r_{s} - \epsilon)}{\hat{\psi}(r_{s})} - 2\mathcal{A}\left(1 + \ln\left(\frac{\epsilon}{a}\right)\right) \right]$$
(3.3)

is a quantity that, similar to Δ' , can be obtained from the ideal outer solution and σ is a constant that defines whether the equilibrium resistivity is uniform ($\sigma=0$) or the equilibrium electric field is uniform ($\sigma=1$). Equation (3.1) can be used to predict the saturated island width, $w_{\rm sat}$, as a function only of the initial equilibrium profiles, by setting the left-hand side to zero and solving for w (notice that, as expected, the solution will not depend on η). This involves solving a single nonlinear algebraic equation and that can be easily done numerically. Figure 4 shows the predicted values of $w_{\rm sat}$ for the same family of equilibria as in § 2, namely for different values of q_0 . We observe that $w_{\rm sat}$ increases with q_0 up to a maximum, $w_{\rm sat}/a \approx 0.15$ at $q_0 \approx 1.3$, and then decreases again towards zero as q_0 approaches $q_s=2$. It is worth noticing that the value of Δ' , which gives a measure of the available energy for reconnection (Furth et~al.~1973), is monotonically increasing with q_0 and thus the final amplitude of the mode is not entirely reflected by the amplitude of Δ' . We also observe that the choice of σ (0 or 1) only introduces minor changes.

4. Nonlinear resistive MHD simulations

The dynamics of the tearing mode, from the initially unstable equilibrium up until the nonlinear saturation of the island, can be calculated by using an initial-value approach. Here, we use the SpeCyl code (Cappello 2004; Bonfiglio *et al.* 2010) to solve the nonlinear

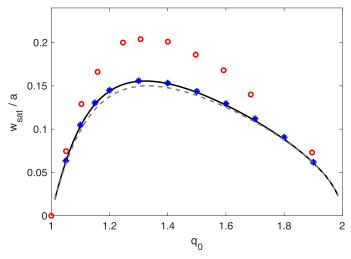


FIGURE 4. Value of w_{sat} vs q_0 as obtained from (3.1) with $\sigma = 1$ (solid black line) and $\sigma = 0$ (dashed grey line). Also shown are the predictions from the initial-value code SpeCyl (blue stars) and from the equilibrium code SPEC (red circles). The initially unstable equilibria are the same as for figure 3.

visco-resistive MHD equations in a cylinder, in the constant-pressure, constant-density approximation. The model equations are, in dimensionless units,

$$\frac{\partial \mathbf{B}}{\partial t} = -\nabla \times \mathbf{E},
\mathbf{E} = \eta \mathbf{j} - \mathbf{v} \times \mathbf{B},
\mathbf{j} = \nabla \times \mathbf{B},
\rho \frac{\mathrm{d}\mathbf{v}}{\mathrm{d}t} = \mathbf{j} \times \mathbf{B} + \rho \nu \nabla^2 \mathbf{v},
\nabla \cdot \mathbf{B} = 0,$$
(4.1)

where E and B are the electric and magnetic fields, j is the current density, v is the plasma velocity, $dv/dt = [\partial v/\partial t + (v \cdot \nabla)v]$ and $\rho = m_i n_0$ is the (constant) mass density. Lengths are normalized to the minor radius a, the plasma particle density to the uniform density n_0 , the magnetic field to the initial value B_0 of the axial magnetic field on axis, the velocity to the Alfvén velocity $v_A = B_0/(\mu_0 m_i n_0)^{1/2}$ and time to the Alfvén time $\tau_A = a/v_A$. In these units, the resistivity η corresponds to the inverse Lundquist number $S^{-1} = \tau_A/\tau_\eta$, and the viscosity v to the inverse viscous Lundquist number (for a scalar kinematic viscosity), $M^{-1} = \tau_A/\tau_v$, where τ_η and τ_v are the resistive time scale and viscous time scale, respectively. The equations are solved in cylindrical geometry (r, θ, z) and periodic boundary conditions are used for both the azimuthal coordinate θ and the axial coordinate z, which has periodicity $2\pi R$. The code uses finite differences in the radial coordinate r and a spectral formulation in the two periodic coordinates θ and z. In the calculations presented in this paper, the Fourier harmonics (m, n) = (2j, j) are retained, with $j = 0, \ldots, 16$. Such large number of helical harmonics is required to properly capture the sharp flow patterns developing around the island separatrix, in particular for high Lundquist numbers. The boundary conditions at the plasma wall, r = a, are $B_r = E_\theta = 0$ (perfect conductor), $E_z = E_0$ (externally imposed, constant, toroidal electric field) and

 $v_{\theta} = v_z = 0$ (no-slip boundary conditions). The rest of the boundary conditions can be derived from Ohm's law. More details about the SpeCyl code and its numerical algorithm can be found in Cappello & Biskamp (1996) and Cappello (2004).

We run SpeCyl for a number of initially unstable equilibria that, as described in § 2, are characterized by q_0 . For each value of q_0 , we consider different values for the Lundquist number from $S = 10^6$ to $S = 10^9$ at fixed magnetic Prandtl number P = S/M = 1, we evolve the system until saturation and measure the island width. Figure 5 illustrates examples of such saturated states by showing a Poincaré section of the magnetic field obtained for $q_0 = 1.3$ (top left) and $q_0 = 1.6$ (bottom left) with $S = 10^8$. For sufficiently high Lundquist number $S > 10^8$, the radial profiles of the perturbed flux function $\hat{\psi}$ show good agreement in the linear growth phase with those produced by the shooting code, and the nonlinear saturated island width becomes independent of S (data not shown). This is consistent with previous analytical and numerical studies (see e.g. Poyé et al. 2014) showing that the saturated tearing mode island width is independent of resistivity and viscosity. We also find very good agreement with the analytical predictions from the nonlinear theory of Arcis for all the considered tearing unstable equilibria (figure 4). In particular, the best agreement with nonlinear theory is found assuming $\sigma = 1$ (i.e. a uniform equilibrium electric field) consistently with the assumption of the resistive MHD code. From the resistive simulations, we also observe that the saturated island is not symmetric with respect to the X-point. More precisely, we can define the asymmetry parameter

$$A_{\text{sym}} = \frac{r_X - r_-}{r_+ - r_X} - 1,\tag{4.2}$$

where r_X is the radial position of the X-point of the island and r_- , r_+ are respectively the smallest and largest radial positions of the island separatrix (which occur at the poloidal position of the O-point). For a symmetric island, $r_X - r_- = r_+ - r_X$ and thus $A_{\rm sym} = 0$. We can evaluate (4.2) from the SpeCyl simulation results and display $A_{\rm sym}$ as a function of q_0 (figure 6). Figure 7 also shows how the individual values of r_X , r_- and r_+ change with q_0 . As soon as $q_0 > 1$, the islands become quickly asymmetrical, $A_{\rm sym} > 0$, and become more and more asymmetric as q_0 approaches 2. We remark that the asymmetric feature of tearing mode islands has been observed in previous investigations and has been suggested to play an important role in determining the saturation amplitude from a modified Rutherford equation approach (Teng *et al.* 2016).

5. Nonlinear equilibrium calculations

We now use the SPEC code to seek stable equilibrium solutions by starting from the unstable equilibria described in § 2. Indeed, in § 2.2 we obtained, for each value of q_0 , the unstable eigenmode $u_{\lambda} \equiv \{r_{l,mn}\}$ that is associated with the unstable (negative) eigenvalue λ of the force-gradient matrix \mathcal{G} . Here, m=2, n=1 and the index $l=1,2,\ldots,N_v-1$ labels the interfaces. Figure 8 shows an example of such an eigenmode. We now perturb the Fourier geometry of the initial equilibrium interfaces by adding a displacement proportional to the eigenmode, namely, we add α u_{λ} with $\alpha > 0$. Notice that, given a certain volume (or equivalently, magnetic flux) enclosed by the two interfaces surrounding the resonant surface, there is a maximum perturbation amplitude beyond which the interfaces would overlap – but this is never exceeded because the perturbation is typically small enough. SPEC then seeks a new equilibrium solution. If α is too small, SPEC snaps back to the initial equilibrium (figure 9). If α is large enough, however, SPEC finds another equilibrium solution (independent of α , see figure 9) that has a helical structure and displays a magnetic island inside the volume containing the resonant surface and

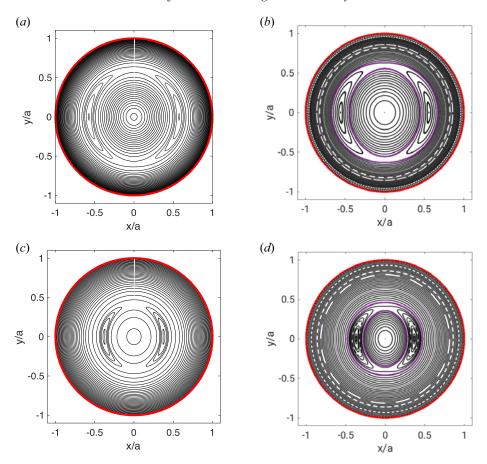


FIGURE 5. Poincaré sections of the magnetic field at $\varphi = 0$ showing the saturated island as obtained from SpeCyl (a,c) and SPEC (b,d) for $q_0 = 1.3$ (a,b) and $q_0 = 1.6$ (c,d). The violet lines on the SPEC Poincaré sections (b,d) indicate the two interfaces encapsulating the resonant volume.

thus where reconnection happens (two examples are shown in figure 5). This approach was successful in slab geometry and showed that the saturation amplitude of the tearing mode is correctly retrieved (Loizu et al. 2020). In that study, the toroidal magnetic flux Ψ_{w} enclosed by the resonant volume, which constrains the maximum achievable island width, was chosen by following a recipe without free parameters that is summarized in § 5.1 for convenience. If the island is expected to be perfectly symmetric around the resonant surface, the value of Ψ_w uniquely determines the initial positions r_+ and r_- of the two interfaces defining the resonant volume, since $\Psi_w = B_z \pi (r_\perp^2 - r_\perp^2)$ and the symmetry argument forces the flux to be equally distributed on each side of the resonant radius r_s . In general, however, there are multiple ways of placing the two interfaces given the same total flux Ψ_w (see figure 10 for an illustration). For this reason, we need an additional constraint that is related to the expected island asymmetry, and that is the subject of § 5.2. Finally, the search for a nearby, lower energy equilibrium state can be done with different constraints, and an appropriate choice must be made by identifying the good quasi-invariants in the process at play. In the case of tearing mode reconnection, it is the net-toroidal-current profile that shall be constrained, and that is the subject of § 5.3. Section 5.4 summarizes

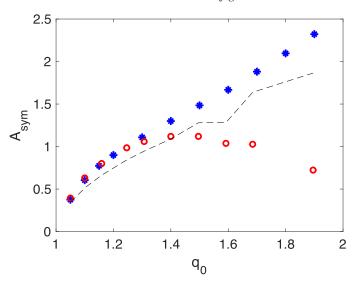


FIGURE 6. Value of A_{sym} as a function of q_0 as obtained from SpeCyl (blue stars) and SPEC (red circles). The dashed line indicates the expected island asymmetry A_{max} obtained by solving by solving (5.4) and that is used to initialize the SPEC equilibrium.

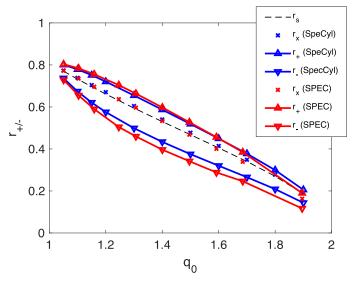


FIGURE 7. Radial positions r_X , r_- and r_+ characterizing the island, as a function of q_0 as obtained from SpeCyl (blue) and SPEC (red). The dashed line indicates the position of the resonant surface.

the results of nonlinear simulations carried out with SPEC to reproduce the saturated tearing mode island width as a function of q_0 .

5.1. Determining Ψ_w

In the limit $\Psi_w \to 0$, the volume available for reconnection vanishes and therefore the maximum island width that is possibly achievable goes to zero. For increasingly large values of Ψ_w , the island is allowed to grow over a larger volume, although it may well

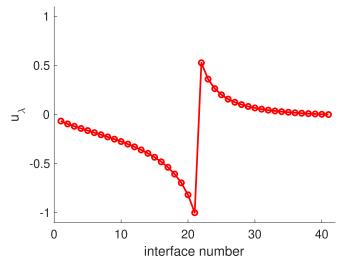


FIGURE 8. Unstable eigenmode profile showing the typical tearing parity around the resonant surface. Here, $q_0 = 1.1$ and $N_v = 41$.

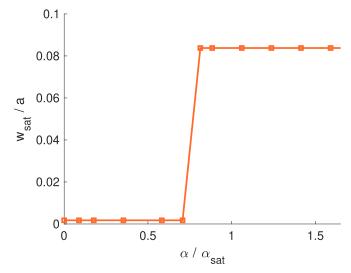


FIGURE 9. Saturated island width obtained from SPEC nonlinear calculations with $q_0 = 1.1$ as a function of the initial perturbation amplitude as measured by α . The normalizing factor α_{sat} represents the actual saturation amplitude of the helical state obtained after the nonlinear calculation. The Fourier resolution is (m, n) = (4, 2).

only occupy a fraction of it. However, if the value of Ψ_w exceeds a certain threshold, Ψ_w^* , the initial equilibrium becomes stable and no island grows at all. This is illustrated in figure 11 for two cases. Obviously, for each q_0 , the saturated island width obtained with SPEC depends on Ψ_w . However, there is a distinct point, which we identify as w_{sat} , and that is the maximum achieved island width, which occurs at around $\Psi_w \approx \Psi_w^*$. We thus apply the following procedure: for each value of q_0 , a linear stability analysis is first performed with SPEC (see § 2.2) and the value of Ψ_w^* is obtained by solving $\lambda(\Psi_w) = 0$. This criterion is reminiscent of the quasi-linear theory of R. White (White *et al.* 1977), in which the

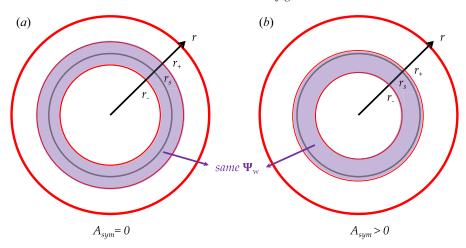


FIGURE 10. An illustration of how the initial positions r_+ and r_- of the interfaces defining the resonant volume can be symmetric (a) or asymmetric (b) with respect to the resonant surface $r = r_s$, while preserving the same enclosed toroidal flux Ψ_w .

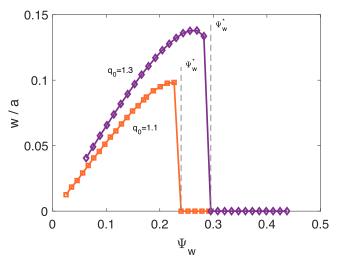


FIGURE 11. Island width, w, obtained from SPEC nonlinear calculations as a function of the enclosed toroidal flux Ψ_w imposed in the resonant volume. Two values of q_0 are shown: $q_0 = 1.1$ (orange squares) and $q_0 = 1.3$ (purple diamonds). The vertical dashed lines indicate the value of Ψ_w^* . Here, the flux is always equally distributed on each side of the resonant surface, and the toroidal-current profile is constrained. The Fourier resolution is low here (m = 2, n = 1).

saturation of a tearing mode occurs at a marginal stability point evaluated as $\Delta'(w) = 0$. We remark that the value of Ψ_w^* obtained with this procedure is independent of the number of volumes for sufficiently large N_v (see figure 12). Then, SPEC is run nonlinearly with $\Psi_w \lesssim \Psi_w^*$ and the obtained value of w_{sat} is extracted from the corresponding saturated state.

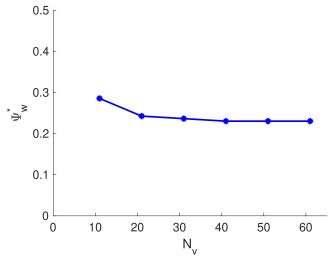


FIGURE 12. Values of Ψ_w^* (obtained by solving $\lambda(\Psi_w) = 0$) as a function of the number of volumes. Here, the equilibrium has $q_0 = 1.1$.

5.2. *Model for the island asymmetry*

The SPEC calculations shown in figure 11 were done by assuming that the flux Ψ_w in the resonant volume is equally distributed on each side of the resonant surface. However, a different choice (as illustrated in figure 10) can substantially modify the nonlinear saturation amplitude and thus the value of $w_{\rm sat}$. In the following, we describe how we can estimate the expected asymmetry of the island and thus constrain accordingly the initial position of the interfaces, r_+ and r_- , around the resonant surface.

We start by writing the total magnetic field as $\mathbf{B} = \mathbf{B}_0(r) + \nabla \psi \times \hat{\mathbf{z}}$ and assume an arbitrarily large perturbation with single helicity, $\psi(r, \zeta) = \hat{\psi}(r) \cos \zeta$, where $\zeta = m\theta - n\varphi$. This perturbation generates an island around the rational surface with $q_s = q(r_s) = m/n$. The X-point of the island is located at $\zeta = 0$ and the O-point of the island is located at $\zeta = \pi$. However, as we show now, this island is not symmetric in the sense of (4.2), namely $A_{\text{sym}} > 0$. The island itself consists of a set of nested magnetic surfaces that are described by a flux function $\chi(r, \zeta)$ such that $\mathbf{B} \cdot \nabla \chi = 0$. One can show that (Fitzpatrick 1995)

$$\chi(r,\zeta) = \int_{r}^{r_s} \left(1 - \frac{q}{q_s}\right) B_\theta \, \mathrm{d}r + \hat{\psi}(r) \cos \zeta, \tag{5.1}$$

and so the magnetic surfaces in the island, and in particular the island separatrix, are surfaces of constant χ . Furthermore, the asymptotic behaviour of $\hat{\psi}(r)$ around the resonant surface can be obtained from the linear ideal outer solutions (see § 6.9 of Wesson 2004) and is given by

$$\hat{\psi}^{\pm}(r) = \psi_s \left[1 + \mathcal{A}(r - r_s) \ln |r - r_s| + A^{\pm}(r - r_s) \right], \tag{5.2}$$

where \pm refers to each side of the resonant surface, ψ_s is the (unknown) amplitude of the perturbation at the resonant surface, \mathcal{A} is given by (3.2a-c) and A^{\pm} is a constant that can be determined numerically by solving for the radial profile of $\hat{\psi}(r)$ outside of the resonant layer. As a matter of fact, the constants A^{\pm} are used to evaluate Δ' and Σ' as defined in

(2.3) and (3.3), since $\Delta' = A^+ - A^-$ and $\Sigma' = A^+ + A^-$. At the separatrix, we have

$$\chi(r_+, \pi) = \chi(r_-, \pi) \equiv \chi_{\text{sep}} \approx \chi(r_s, 0) = \psi_s, \tag{5.3}$$

and hence combining (5.1) and (5.2) into (5.3) we obtain a constraint for (r_+, r_-)

$$g(r_{+}) = g(r_{-}),$$
 (5.4)

where

$$g(r) = \frac{\int_{r}^{r_s} \left(1 - \frac{q}{q_s}\right) B_{\theta} \, \mathrm{d}r}{2 + \mathcal{A}(r - r_s) \ln|r - r_s| + A^{\pm}(r - r_s)},\tag{5.5}$$

is a known function of the equilibrium. Equation (5.4) can be combined with the constraint for the toroidal flux, $\Psi_w = \pi B_{z0}(r_+^2 - r_-^2)$, to solve for the positions r_+ and r_- of the interfaces that encapsulate the resonant volume. The values obtained for r_+ and r_- can then be used to initialize the SPEC equilibrium interfaces. Given this initial geometry, one can show that the expected (maximum) value of the island asymmetry at saturation is

$$A_{\text{max}} = 2\left(\frac{r_s - r_-}{r_+ - r_s} - 1\right). \tag{5.6}$$

The value of A_{max} obtained by solving (5.4) for different values of q_0 is shown in figure 6 and the agreement with the actual saturated island asymmetry obtained by SpeCyl is quite remarkable. Notice that this prediction is merely based on linear theory.

5.3. Constraining the current profile

Magnetic reconnection is a process through which magnetic energy is converted into kinetic and thermal energy. If the process is fast enough, the magnetic helicity is well conserved (Berger 1999), and this happens for example during sawteeth in tokamaks (Heidbrink & Dang 2000), which occur on quasi-ideal time scales. Resistive tearing modes, however, are instabilities that promote magnetic reconnection at a rate that is fast when compared with the resistive time scale τ_{η} , but slow when compared with the Alfvénic time scale τ_{A} . The reconnection process is then too slow for the magnetic helicity to be a good invariant. Nevertheless, it can be shown that the net toroidal current is a flux function at saturation and is given by (Escande & Ottaviani 2004; Militello & Porcelli 2004)

$$j_z(\psi) = \langle j_{\text{eq}} \rangle_{\psi}, \tag{5.7}$$

where $\langle \cdot \rangle_{\psi}$ is the average over a flux surface and $\psi(r, \theta, \varphi)$ is the saturated flux function. Equation (5.7) implies the quasi-invariance of the current profile, the invariance being exact in the limit of small island width. We can thus exploit this constraint on the current profile and run SPEC nonlinear equilibrium calculations by constraining the current in each volume, I_{vol} , to remain constant, and the surface current on each interface, I_{surf} , to remain zero, thereby ensuring that the current profile remains smooth.

5.4. Nonlinear simulation results

For each value of q_0 , we thus proceed as follows: (i) perform a linear stability analysis with SPEC to find the unstable eigenmode u_{λ} and the value of Ψ_w^* (§ 5.1); (ii) use (5.4) to infer the values of (r_+, r_-) (§ 5.2); (iii) perturb the equilibrium interfaces according to u_{λ} and use SPEC to seek a new equilibrium by constraining the current profile (§ 5.3); (iv)

measure the width of the island in the resonant volume of the newly found equilibrium. We remark that this procedure has no free parameters. Figure 5 shows two examples of saturated states obtained with SPEC for $q_0 = 1.3$ (top right) and $q_0 = 1.6$ (bottom right), which qualitatively look very similar to the ones obtained with SpeCyl (top left and bottom left). Figure 4 shows the saturated island width w_{sat} obtained with SPEC as a function of q_0 . The qualitative shape and order of magnitude of the expected function $w_{\rm sat}(q_0)$ is reproduced. For small enough islands, the agreement with the predictions from nonlinear theory and resistive MHD simulations is remarkable. The agreement remains good up to island sizes of the order of 10% of the minor radius. For increasingly large islands, the equilibrium calculations overestimate the saturation amplitude from nonlinear theory and resistive MHD simulations (the maximum relative error is approximately 25 %). This might be due to the fact that these calculations assume a flat current profile inside the island. Furthermore, the initial positioning of the interfaces around the rational surface, which is determined by solving (5.4), has some approximations, namely that the nonlinear perturbed flux function around the resonant surface has a single helicity and a radial structure given by the linear solution. As a matter of fact, we can measure a posteriori the positions r_X , r_- and r_+ that characterize the island at saturation in SPEC, and compare them with those obtained with SpeCyl for different values of q_0 . This is shown in figure 7. In particular, we see that the sign of the asymmetry is well retrieved, namely the property that $r_X - r_- > r_X - r_+$. However, the exact value of A_{sym} , (4.2), is quite sensitive to small errors in the values of r_X , r_- and r_+ , and a good agreement is only observed for $q_0 \lesssim 1.3$ (see figure 6). We also compare the profiles of each component of B and j at saturation as obtained from SPEC and SpeCyl. Figures 13 and 14 show this comparison for the case $q_0 = 1.3$, for which the agreement in the island width is the least good. Even for this case, we still observe that SPEC is capable of reproducing (both qualitatively and quantitatively) the structure of the saturated tearing mode. We can also compare the axial current profile at saturation, $j_r(r)$, taken through the O-point and X-point of the island (figure 15). In particular, we indeed see that in SpeCyl the current significantly flattens, but not perfectly, across the island.

Finally, we would like to remark that a convergence analysis has been carried out for the SPEC simulations presented herein. Increasing further the number $N_v = 41$ of volumes does not significantly change the values of w_{sat} (data not shown). Similarly, no significant changes are obtained with a further increase in the order of the radial basis functions (Tschebyshev polynomials) used to represent the magnetic vector potential in the volumes. The Fourier resolution, however, seems to play a more important role. Even if the dominant Fourier mode in the saturated island is the (m, n) = (2, 1) mode and higher harmonics (m,n)=(2j,j), with $j\in\mathbb{N}$, are present with exponentially decreasing amplitude, the values of w_{sat} can strongly depend on the Fourier resolution used in SPEC, as illustrated in figure 16. At a resolution of (m, n) = (10, 5), which is the one used to obtain the results in figure 4 and already implies describing 116 different Fourier modes (Hudson et al. 2020), the relative changes in w_{sat} due to a further increase in Fourier resolution become lower than 5 %. It is also important to note that, for q_0 close to 2, the resonant surface is close to the magnetic (and coordinate) axis, and that makes it harder for SPEC to robustly find a solution, mainly because of the singularity in the coordinates at r=0. Furthermore, the SPEC code is still numerically fragile in the sense that the Newton-like method employed to find force balance might not be able to converge depending on the initial guess. The main result of this paper is thus the evidence that an equilibrium approach is capable of predicting, to a certain level of accuracy, the saturated state of tearing mode islands in a cylindrical tokamak. The potential of carrying out these predictions very fast is still to be shown by improving the numerical algorithm in SPEC. Nevertheless, when SPEC is

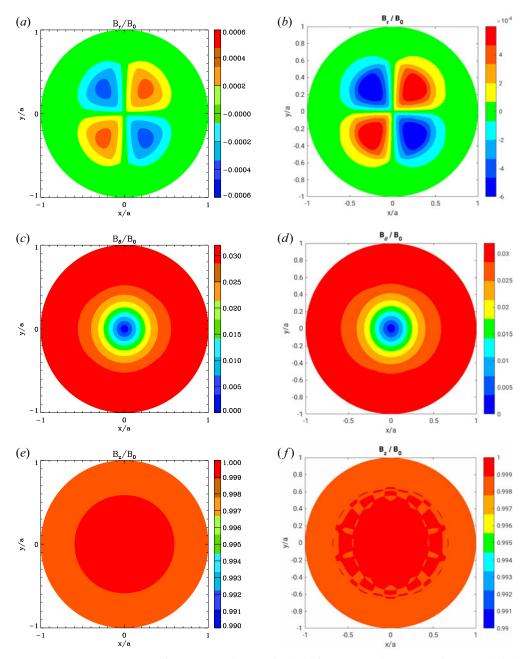


FIGURE 13. Components of **B** at saturation as obtained from SpeCyl (a,c,e) and SPEC (b,d,f) for $q_0 = 1.3$. For each component, the color scales are the same between SpeCyl and SPEC.

able to find a solution, it does so in \sim 3 CPU hours, while the time-dependent resistive MHD calculations performed with SpeCyl can take hours to days, mainly depending on the value of the Lundquist number S, which strongly affects the simulation time required to reach saturation, and on the number of Fourier harmonics included in the calculation. The SpeCyl predictions are, however, more accurate, as was shown in figure 4.

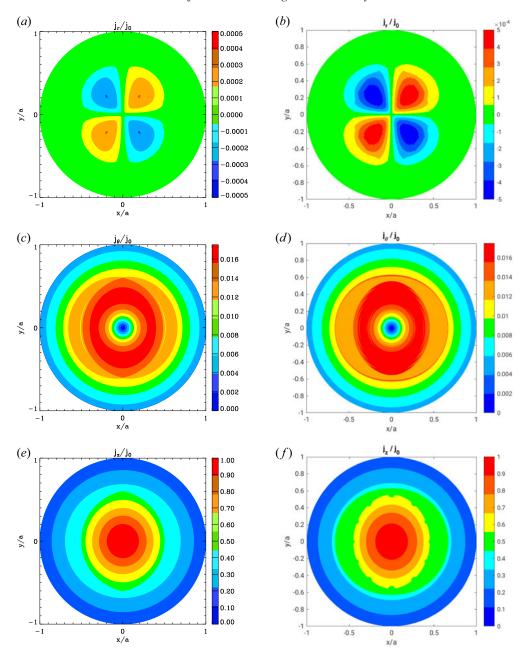


FIGURE 14. Components of j at saturation as obtained from SpeCyl (a,c,e) and SPEC (b,d,f) for $q_0=1.3$. For each component, the color scales are the same between SpeCyl and SPEC.

6. Conclusions

In this paper we have investigated the nonlinear saturation of resistive tearing modes in a zero-pressure cylindrical tokamak. We have validated the results of nonlinear visco-resistive MHD simulations against an exact nonlinear saturation theory, showing excellent agreement for a wide range of equilibria. Furthermore, we have shown that an

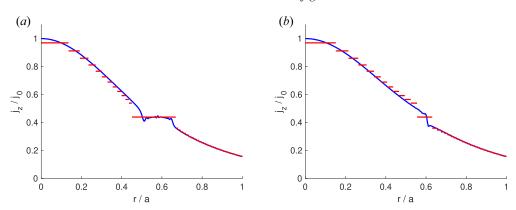


FIGURE 15. Comparison of the axial current profiles at saturation (red is SPEC, blue is SpeCyl) taken through the O-point (*a*) and X-point (*b*) of the island.

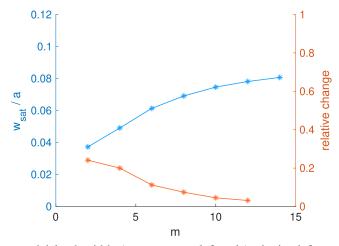


FIGURE 16. Saturated island width (upper curve, left axis) obtained from SPEC nonlinear calculations with $q_0 = 1.05$ as a function of the poloidal Fourier resolution m. The toroidal Fourier resolution is always n = m/2. The relative error defined here as $(w_{\text{sat}}(m+1) - w_{\text{sat}}(m))/w_{\text{sat}}(m)$ is also shown (lower curve, right axis).

equilibrium approach can be used to predict, without free parameters, the saturated state of these modes to a certain degree of accuracy, with a very good agreement for sufficiently small islands. The natural next step is to perform similar tearing mode saturation studies in toroidal geometries. In stellarators, classical tearing modes have been observed (Bartlett *et al.* 1980) and modelled (Nikulsin *et al.* 2022), and we plan on investigating them with the SPEC code. In tokamaks, neoclassical tearing modes play a crucial role in the potential trigger of plasma disruptions, and we plan on extending this approach with the addition of the bootstrap current effects, which are already present in the SPEC code (Baillod *et al.* 2021, 2023). There are also toroidal corrections to classical tearing modes in tokamaks (Glasser, Greene & Johnson 1975), which have already been reproduced linearly with the SPEC code (Kumar *et al.* 2023). We plan on verifying whether the effect of these corrections on the nonlinear saturation of classical tearing modes can be retrieved with an equilibrium approach as well. Finally, the effect of rotation on tearing modes could also

be studied by exploiting recent extensions of the stepped-pressure equilibrium model that include the presence of flows (Qu *et al.* 2020).

Acknowledgements

The authors acknowledge useful discussions with A. Bhattacharjee, S. Cappello, A. Cerfon, D. Escande, P. Helander, S. Hudson and Z. Qu.

Editor P. Helander thanks the referees for their advice in evaluating this article.

Funding

This work has been carried out within the framework of the EUROfusion Consortium, partially funded by the European Union via the Euratom Research and Training Programme (Grant Agreement No. 101052200 – EUROfusion). The Swiss contribution to this work has been funded by the Swiss State Secretariat for Education, Research and Innovation (SERI). Views and opinions expressed are, however, those of the author(s) only and do not necessarily reflect those of the European Union, the European Commission or SERI. Neither the European Union nor the European Commission nor SERI can be held responsible for them. This research was also supported by a grant from the Simons Foundation (1013657, JL).

Declaration of interests

The authors report no conflict of interest.

Data availability statement

The data that support the findings of this study are available from the authors upon reasonable request.

Appendix A

Here, we show that, in a cylindrical tokamak geometry, the stability of stepped-pressure equilibria can be calculated from the eigenvalues of the force-gradient matrix \mathcal{G} , whose coefficients measure the variation in the force on each interface, $f_l = [[B^2/2\mu_0]]_l$, with respect to perturbations in the geometry of the ideal interfaces, $r_{l,mn}$, with the constraint of fixed magnetic helicity and fluxes in each volume (§ 2.2).

Consider the energy functional

$$F = W - \sum_{l} \mu_{l}(K_{l} - K_{l,o}), \tag{A1}$$

where

$$W = \sum_{l} \int_{V_{l}} \frac{B^{2}}{2\mu_{0}} \, \mathrm{d}V, \tag{A2}$$

is the total plasma potential energy and μ_l are Lagrange multipliers defined to enforce the constraint on the magnetic helicity $K_l = K_{l,o}$ in each volume l. A variational calculation on F that accounts for arbitrary variations in the magnetic field, $\delta \mathbf{B} = \nabla \times \delta \mathbf{A}$ (except for the boundary condition $\mathbf{B} \cdot \mathbf{n} = 0$ and the flux constraints, which can be enforced using

the gauge freedom), and interface geometry, $\delta x \equiv \{\delta r_{l,mn}\}$, gives

$$\delta F = \sum_{l} \int_{V_{l}} (\nabla \times \mathbf{B} - \mu_{l} \mathbf{B}) \cdot \delta \mathbf{A} \, dV + \sum_{l} \int_{\partial V_{l}} f_{l} \boldsymbol{\xi}_{l} (\delta \mathbf{x}) \cdot d\mathbf{s}, \tag{A3}$$

where ξ_l is the displacement vector of the interface l, which depends on δx . At equilibrium, $\delta F = 0$, $\nabla \times B = \mu_l B$ in each volume and $f_l = 0$ on each interface. Notice that the force f_l is a function of the position on the surface describing the interface, and thus can be described, like the interface geometry, in terms of its Fourier spectrum coefficients $f_{l,mn}$, which must all be zero at equilibrium. In general, the force $f \equiv \{f_{l,mn}\}$ is non-zero and depends on the interface geometry $x \equiv \{r_{l,mn}\}$, i.e. f = f(x). We can define the force-gradient matrix as the derivative matrix \mathcal{G} with coefficients

$$\mathcal{G}_{ij}(\mathbf{x}) = \frac{\partial f_i}{\partial x_j}(\mathbf{x}),\tag{A4}$$

and where the derivatives are taken by preserving the constraints (fluxes and helicities) fixed. The matrix \mathcal{G} is square if the number of degrees of freedom (Fourier harmonics) to describe f and x are the same.

Let us now consider a perturbation from an equilibrium state $\delta x = x - x_{\rm eq}$, such that the new state satisfies a Beltrami equation with the same helicities as for the equilibrium field, but now $f(x) \neq 0$. While $\delta F = 0$ around the equilibrium point $x_{\rm eq}$, the variation of the energy functional around x is

$$\delta F = -\sum_{l} \int_{\partial V_{l}} f_{l} \, \boldsymbol{\xi}_{l}(\delta \boldsymbol{x}) \cdot \, \mathrm{d}\boldsymbol{s}, \tag{A5}$$

with the minus sign due to the sign convention for δx . Now, we can expand f_l around x_{eq} using the fact that $f_l(x_{eq}) = 0$, and write

$$\delta F = -\sum_{l} \int_{\partial V_{l}} \left(\frac{\partial f_{l}}{\partial \mathbf{x}} \Big|_{\mathbf{x}_{eq}} \cdot \delta \mathbf{x} \right) \boldsymbol{\xi}_{l}(\delta \mathbf{x}) \cdot d\mathbf{s}. \tag{A6}$$

Given the expansion we have performed, one finds that the infinitesimal variation of potential energy $\delta W \equiv W(x) - W(x_{eq})$ is thus

$$\delta W = \frac{1}{2} \sum_{l} \int_{\partial V_{l}} \left(\frac{\partial f_{l}}{\partial \mathbf{x}} \Big|_{\mathbf{x}_{eq}} \cdot \delta \mathbf{x} \right) \boldsymbol{\xi}_{l}(\delta \mathbf{x}) \cdot d\mathbf{s}. \tag{A7}$$

We now show that in cylindrical geometry, the sign of δW in (A7) is given by the sign of the eigenvalues of the matrix \mathcal{G} . Indeed, in cylindrical geometry, we have that the position of an interface is

$$\mathbf{r}_{l}(\theta,\varphi) = r_{l}(\theta,\varphi)\cos\theta\,\hat{\mathbf{i}} + r_{l}(\theta,\varphi)\sin\theta\,\hat{\mathbf{j}} + R\varphi\,\hat{\mathbf{k}},\tag{A8}$$

and we can expand in Fourier series the function $r_l(\theta, \varphi)$

$$r_l(\theta, \varphi) = \sum_k r_{l,k} \cos(\alpha_k),$$
 (A9)

where $\alpha_k = m_k \theta - n_k \varphi$, with k labelling all the Fourier modes in the expansion. The displacement of an interface is then given by

$$\boldsymbol{\xi}_{l} = \delta \boldsymbol{r}_{l}(\theta, \varphi) = \sum_{k} \delta \boldsymbol{r}_{l,k} \cos{(\alpha_{k})} (\cos{\theta} \,\hat{\boldsymbol{i}} + \sin{\theta} \,\hat{\boldsymbol{j}}), \tag{A10}$$

which expresses the relation between ξ_l and certain components of δx . Meanwhile, the differential surface element, ds, projected on the displacement vector, ξ_l , gives

$$\boldsymbol{\xi} \cdot d\boldsymbol{s} = Rr_l(\theta, \varphi) \sum_k \delta r_{l,k} \cos(\alpha_k) d\theta d\varphi, \tag{A11}$$

and we can thus express (A7) as

$$\delta W = \frac{1}{2} \sum_{l} \int_{\partial V_{l}} \left(\frac{\partial f_{l}}{\partial x} \Big|_{x_{eq}} \cdot \delta x \right) \boldsymbol{\xi}_{l}(\delta x) \cdot ds$$
 (A12)

$$= \frac{1}{2} \sum_{l} \int_{\partial V_{l}} \left(\frac{\partial f_{l}}{\partial x} \Big|_{x_{eq}} \cdot \delta x \right) Rr_{l}(\theta, \varphi) \sum_{k} \delta r_{l,k} \cos(\alpha_{k}) d\theta d\varphi$$
(A13)

$$= \frac{1}{2} \sum_{l} \sum_{k} \sum_{k'} \int_{\partial V_{l}} \left(\frac{\partial f_{l,k'}}{\partial x} \Big|_{x_{eq}} \cdot \delta x \right) \cos(\alpha_{k'}) \delta r_{l,k} \cos(\alpha_{k}) R r_{l}(\theta, \varphi) d\theta d\varphi$$
 (A14)

$$= \frac{R}{2} \sum_{l} \sum_{k} \sum_{k'} \left(\frac{\partial f_{l,k'}}{\partial \mathbf{x}} \Big|_{\mathbf{x}_{eq}} \cdot \delta \mathbf{x} \right) \delta r_{l,k} \int_{\partial V_l} \cos(\alpha_k) r_l(\theta, \varphi) \, d\theta \, d\varphi \qquad (A15)$$

$$= \frac{R}{2} \sum_{l} \sum_{k} \sum_{k'} \sum_{k''} \left(\frac{\partial f_{l,k'}}{\partial x} \Big|_{x_{eq}} \cdot \delta x \right) \delta r_{l,k} r_{l,k''} \int_{\partial V_l} \cos(\alpha_{k'}) \cos(\alpha_{k}) \cos(\alpha_{k''}) d\theta d\phi$$
(A16)

$$= \frac{R}{2} \sum_{l} \sum_{k'} \sum_{k''} \left(\frac{\partial f_{l,k'}}{\partial x} \Big|_{x_{eq}} \cdot \delta x \right) \delta r_{l,k} r_{l,k''} \mathcal{I}_{kk'k''}, \tag{A17}$$

where

$$\mathcal{I}_{kk'k''} = \oint \oint \cos(\alpha_k) \cos(\alpha_{k'}) \cos(\alpha_{k''}) d\theta d\varphi. \tag{A18}$$

Finally, assuming that the equilibrium is axisymmetric, $r_{l,k} = r_{l,0}\delta_{k,0}$, and that δx is an eigenvector of \mathcal{G} with eigenvalue λ , $\mathcal{G} \cdot \delta x = \lambda \delta x$, we have

$$\delta W = \frac{R}{2} \sum_{l} \sum_{k'} \sum_{k'} \left(\frac{\partial f_{l,k'}}{\partial x} \Big|_{x_{\text{eq}}} \cdot \delta x \right) \delta r_{l,k} r_{l,0} \mathcal{I}_{kk'0}$$
 (A19)

$$= \lambda \frac{R}{2} \sum_{l} \sum_{k'} \delta r_{l,k'} \delta r_{l,k} \ r_{l,0} \mathcal{I}_{kk'0}$$
(A20)

$$= \lambda \pi^2 R \sum_{l} \sum_{k'} \sum_{k'} \delta r_{l,k'} \delta r_{l,k} r_{l,0} \delta_{k,k'}$$
(A21)

$$= \lambda \pi^2 R \sum_{l} \sum_{k} \delta r_{l,k}^2 \, r_{l,0}, \tag{A22}$$

and hence the sign of δW is given by the sign of λ . Hence, the stability of an axisymmetric equilibrium can therefore be inferred from the (sign of the) eigenvalues of \mathcal{G} .

REFERENCES

- ARCIS, N., ESCANDE, D.F. & OTTAVIANI, M. 2006 Rigorous approach to the nonlinear saturation of the tearing mode in cylindrical and slab geometry. *Phys. Plasmas* 13 (5), 052305.
- BAILLOD, A., LOIZU, J., QU, Z., ARBEZ, H.P. & GRAVES, J.P. 2023 Equilibrium beta-limits dependence on bootstrap current in classical stellarators. *Accepted in J. Plasma Phys*.
- BAILLOD, A., LOIZU, J., QU, Z.S., KUMAR, A. & GRAVES, J.P. 2021 Computation of multi-region, relaxed magnetohydrodynamic equilibria with prescribed toroidal current profile. *J. Plasma Phys.* **87** (4), 905870403.
- BARTLETT, D.V., CANNICI, G., CATTANEI, G., DORST, D., EISNER, A., GRIEGER, G., HACKER, H., HOW, J., JÄCKEL, H., JAENICKE, R., *et al.* 1980 Stabilization of the (2, 1) tearing mode and of the current disruption in the W VII–A stellarator. *Nucl. Fusion* **20** (9), 1093–1100.
- BERGER, M.A. 1999 Introduction to magnetic helicity. Plasma Phys. Control. Fusion 41, 166-176.
- BONFIGLIO, D., CHACÓN, L. & CAPPELLO, S. 2010 Nonlinear three-dimensional verification of the SPECYL and PIXIE3D magnetohydrodynamics codes for fusion plasmas. *Phys. Plasmas* 17 (8), 082501.
- CAPPELLO, S. 2004 Bifurcation in the MHD behaviour of a self-organizing system: the reversed field pinch (RFP). *Plasma Phys. Control. Fusion* **46** (12 B), B313.
- CAPPELLO, S. & BISKAMP, D. 1996 Reconnection processes and scaling laws in reversed field pinch magnetohydrodynamics. *Nucl. Fusion* **36** (5), 571–581.
- COOPER, W.A., GRAVES, J.P., POCHELON, A., SAUTER, O. & VILLARD, L. 2010 Tokamak magnetohydrodynamic equilibrium states with axisymmetric boundary and a 3D helical core. *Phys. Rev. Lett.* **105** (3), 3–6.
- COOPER, W.A., GRAVES, J.P., SAUTER, O., CHAPMAN, I.T., GOBBIN, M., MARRELLI, L., MARTIN, P., PREDEBON, I. & TERRANOVA, D. 2011 Magnetohydrodynamic properties of nominally axisymmetric systems with 3D helical core. *Plasma Phys. Control. Fusion* **53** (7), 074008.
- DENNIS, G.R., HUDSON, S.R., DEWAR, R.L. & HOLE, M.J. 2013a The infinite interface limit of multiple-region relaxed magnetohydrodynamics. *Phys. Plasmas* **20** (3), 032509.
- DENNIS, G.R., HUDSON, S.R., TERRANOVA, D., FRANZ, P., DEWAR, R.L. & HOLE, M.J. 2013b Minimally constrained model of self-organized helical states in reversed-field pinches. *Phys. Rev. Lett.* 111 (5), 1–5.
- ESCANDE, D.F. & OTTAVIANI, M. 2004 Simple and rigorous solution for the nonlinear tearing mode. *Phys. Lett.* A **323**, 278–284.
- FITZPATRICK, R. 1995 Helical temperature perturbations associated with radially asymmetric magnetic island chains in tokamak plasmas. *Phys. Plasmas* 2, 825.
- FREIDBERG, J.P. 2014 Ideal MHD. Cambridge University Press.
- FURTH, H.P., RUTHERFORD, P.H. & SELBERG, H. 1973 Tearing mode in the cylindrical tokamak. *Phys. Fluids* **16** (7), 1054–1063.
- GLASSER, A.H., GREENE, J.M. & JOHNSON, J.L. 1975 Resistive instabilities in general toroidal plasma configurations. *Phys. Fluids* **18** (7), 875.
- HEIDBRINK, W.W. & DANG, T.H. 2000 Magnetic helicity is conserved at a tokamak sawtooth crash. *Plasma Phys. Control. Fusion* **42**, L31–L36.
- HUDSON, S.R., DEWAR, R.L., DENNIS, G., HOLE, M.J., McGANN, M., VON NESSI, G. & LAZERSON, S. 2012 Computation of multi-region relaxed magnetohydrodynamic equilibria. *Phys. Plasmas* 19 (11), 112502.
- HUDSON, S.R., LOIZU, J., ZHU, C., QU, Z.S., NÜHRENBERG, C., LAZERSON, S., SMIET, C.B. & HOLE, M.J. 2020 Free-boundary MRxMHD equilibrium calculations using the stepped-pressure equilibrium code. *Plasma Phys. Control. Fusion* **62** (8), 084002.
- KLEINER, A., GRAVES, J.P., BRUNETTI, D., COOPER, W.A., MEDVEDEV, S., MERLE, A. & WAHLBERG, C. 2019 Current and pressure gradient triggering and nonlinear saturation of lown edge harmonic oscillations in tokamaks. *Plasma Phys. Control. Fusion* 61 (8), 084005.

- KUMAR, A., LOIZU, J., HOLE, M.J., QU, Z., HUDSON, S.R. & DEWAR, R.L. 2023 On the relationship between the multi-region relaxed variational principle and resistive inner layer theory. *Plasma Phys. Control. Fusion* **65** (7), 075004.
- KUMAR, A., NÜHRENBERG, C., QU, Z., HOLE, M.J., DOAK, J., DEWAR, R.L., HUDSON, S.R., LOIZU, J., ALEYNIKOVA, K., BAILLOD, A., et al. 2022 Nature of ideal MHD instabilities as described by multi-region relaxed MHD. *Plasma Phys. Control. Fusion* **64** (6), 065001.
- Kumar, A., Qu, Z., Hole, M.J., Wright, A.M., Loizu, J., Hudson, S.R., Baillod, A., Dewar, R.L. & Ferraro, N.M. 2021 Computation of linear MHD instabilities with the multi-region relaxed MHD energy principle. *Plasma Phys. Control. Fusion* **63** (4), 045006.
- LA HAYE, R.J. 2006 Neoclassical tearing modes and their control. Phys. Plasmas 13 (5), 055501.
- LOIZU, J., HUANG, Y.-M., HUDSON, S.R., BAILLOD, A., KUMAR, A. & QU, Z.S. 2020 Direct prediction of nonlinear tearing mode saturation using a variational principle. *Phys. Plasmas* 27 (7), 070701.
- LOIZU, J. & HUDSON, S.R. 2019 Multi-region relaxed magnetohydrodynamic stability of a current sheet. *Phys. Plasmas* **26** (3), 030702.
- MILITELLO, F. & PORCELLI, F. 2004 Simple analysis of the nonlinear saturation of the tearing mode. *Phys. Plasmas* 11, L13.
- NIKULSIN, N., RAMASAMY, R., HOELZL, M., HINDENLANG, F., STRUMBERGER, E., LACKNER, K. & GÜNTER, S. 2022 JOREK3D: an extension of the JOREK nonlinear MHD code to stellarators. *Phys. Plasmas* **29** (6), 063901.
- POYÉ, A., AGULLO, O., SMOLYAKOV, A., BENKADDA, S. & GARBET, X. 2014 Dynamics of magnetic islands in large Δ' regimes. *Phys. Plasmas* **21** (2), 020705.
- Qu, Z.S., Dewar, R.L., Ebrahimi, F., Anderson, J.K., Hudson, S.R. & Hole, M.J. 2020 Stepped pressure equilibrium with relaxed flow and applications in reversed-field pinch plasmas. *Plasma Phys. Control. Fusion* **62** (5), 054002.
- RUTHERFORD, P.H. 1973 Nonlinear growth of the tearing mode. Phys. Fluids 16 (11), 1903–1908.
- TENG, Q., FERRARO, N., GATES, D.A., JARDIN, S.C. & WHITE, R.B. 2016 Nonlinear asymmetric tearing mode evolution in cylindrical geometry. *Phys. Plasmas* 23 (10), 102515.
- WESSON, J. 2004 Tokamaks. Clarendon Press.
- WHITE, R.B. 2014 The Theory of Toroidally Confined Plasmas. Imperial College Press.
- WHITE, R.B., MONTICELLO, D.A., ROSENBLUTH, M.N. & WADDELL, B.V. 1977 Saturation of the tearing mode. *Phys. Fluids* **20**, 800.
- WRIGHT, A.M., KIM, P., FERRARO, N.M. & HUDSON, S.R. 2022 Modelling the nonlinear plasma response to externally applied three-dimensional fields with the Stepped Pressure Equilibrium Code. *J. Plasma Phys.* 88 (5), 905880508.